

RED-IMPACT – Progress report

IMPACT OF PARTITIONING, TRANSMUTATION AND WASTE **REDUCTION** TECHNOLOGIES ON THE FINAL NUCLEAR WASTE DISPOSAL

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**Representing the Red–Impact Project : Impact of Partitioning,
Transmutation and Waste Reduction Technologies on the Final Nuclear Waste
Disposal.**

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Author's Institutions

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- Compagnie Générale de Matières Nucléaires, Paris, France
- Commissariat à l'Energie Atomique, Cadarache, France
- Studiecentrum voor Kernenergie - Centre d'Etude de l'Energie Nucléaire, Mol, Belgium
- Nexia Solutions, Sellafield, UK
- Forschungszentrum Juelich GmbH, Germany
- + 17 other members of RED-IMPACT

Outline

- Red-Impact Partners
- Objectives of Red-Impact
- Project Structure
- Studied scenarios
- Assumptions
- Indicators
- Results and Conclusions
- Added on values

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Partners of Red-Impact

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Partners of Red-Impact

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2 subcontractors:

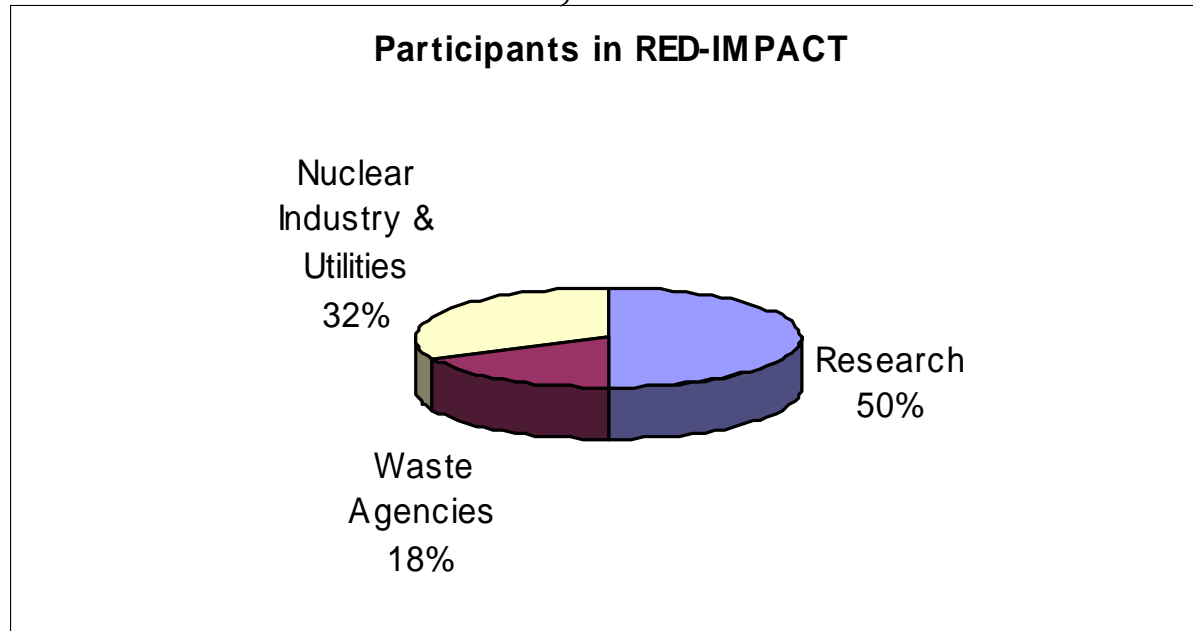
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DECOM (Slovakia)

Partners of RED-IMPACT

**A unique consortium of research institutes, waste agencies
and industrial partners**

23 partners + 2 subcontractors, 11 countries



RED-IMPACT participants



RED-IMPACT website : <http://www.red-impact.proj.kth.se/>

The objectives of RED-IMPACT project – Why do we want to do transmutation:

- Assess the effects of P&T on geological disposal and waste management.
- Assess Economic, Environmental and Societal Costs/benefits of P&T.
- Disseminate results of the study to stakeholders (scientific, general public and decision makers) and get feedback during the study.
- Iterate and refine the work based on stake-holders' feedback to achieve full impact of this study on the implementation of the waste management policy of the European Community.

Red-Impact: six workpackages

- **WP1: Review** of waste management and transmutation strategies, selection of fuel cycles scenarios
- **WP2: Feasibility of the industrial deployment** of selected scenarios and their impact on waste management
- **WP3: Assessment of waste streams**, waste features, leach resistance, heat generation, reprocessing capability etc for selected fuel cycles.
- **WP4: Assessment of the benefits and costs of P&T/C** in advanced fuel cycles for waste management and geological disposal.
- **WP5: Economic, environmental and societal assessment** of fuel cycle strategies
- **WP6: Synthesis and dissemination of results** to stakeholders



**European
Commission**

Project Committee:
Project Participant Meeting
(Consortium meeting)
At least twice a year, each
participant has one vote

RED-IMPACT
Co-ordinator: W. Gudowski
Co-coordinator: R. Odoj

Red-Impact Structure

Project Management Team:
Coordinator, co-coordinator
WP-leaders
Meetings: at least twice a year

WP1 – leader
E. Gonzalez

WP2 – leader
D. Greneche

WP3 – leader
L. Boucher

W4 – leader
J. Marivoet

WP5 – leader
C. Zimmerman

WP6 – leader
W. von Lensa

WP1
Task leaders &
participants

WP2
Task leaders &
participants

WP3
Task leaders &
participants

WP4
Task leaders &
participants

WP5
Task leaders &
participants

WP6
Task leaders &
participants



Set of fuel cycles to assess

Starting point: NEA/OECD report: "Accelerator-Driven Systems (ADS) and Fast Reactors (FR) in Advanced Nuclear Fuel Cycles. A Comparative Study" (2002).

The fuel options taken into account were:

- *the standard reference UO_2 fuel with a ^{235}U enrichment 4.2% with a final average burn-up of 50 GWd/tHM;*
- *mixed oxide fuels (UO_2/PuO_2), with a high burn-up of 50 GWd/tHM;*
- *high Pu content MOX fuel (~ 30 w%) in fast reactors, with a burn-up of 150 GWd/tHM;*
- *mixed Th/Pu fuels with a burn-up of 60 GWd/tHM or inert matrix fuel (IMF)-Pu fuel (instead of thorium), where the inert matrix can be based in the CER-CER concept (Pu diluted in ceramic material) or in the CER-MET concept (takes advantage of higher conductivity of metallic matrices);*
- *the mixture of MA inside MOX fuels or the use of target rods diluted into an inert of partially fertile matrix;*
- *the coated particles, with a fissile kernel and several layers of pyrolytic carbon as pressure resistant envelope and gas diffusion barriers, with a burn-up such as about 600 GWd/tHM.*
- *The chemical reprocessing options considered were the aqueous partitioning system (PUREX, TRUEX, UREX, THOREX, DIAMEX and SANEX processes) and the pyrochemical partitioning system, able to handle fuels with high content in Pu and MA.*

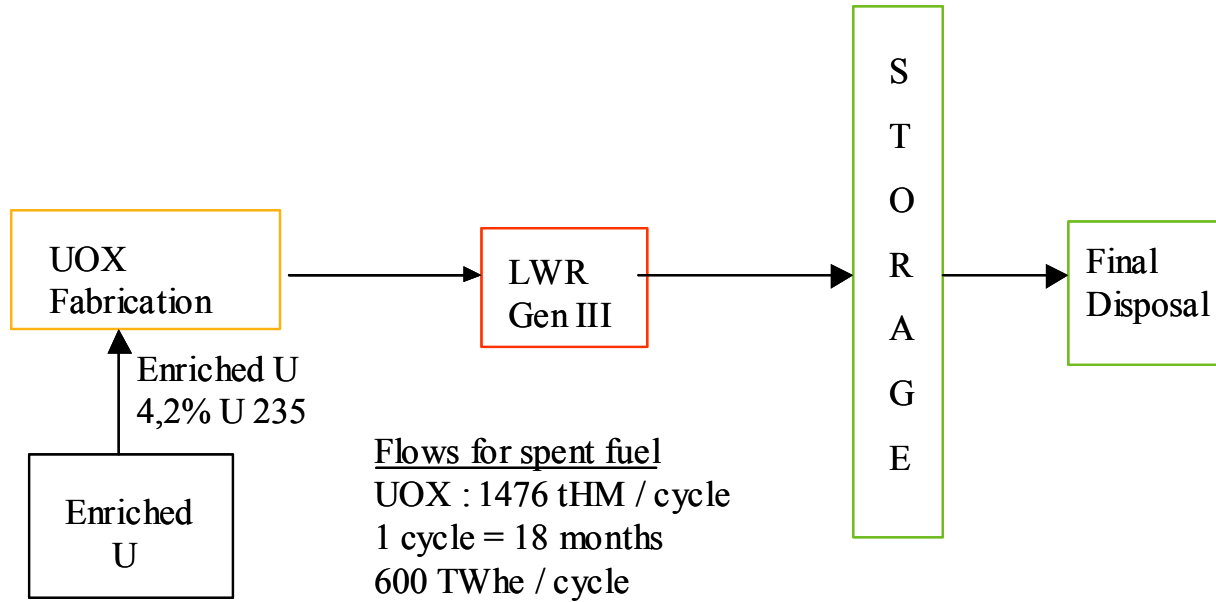
Six basis scenarios are considered for the evaluations

- Three “industrial” scenarios:
 - A1. Reference scenario. Open cycle
 - A2. Near term scenario. Plutonium single recycling in LWR with standard MOX
 - A3. Fast reactor with infinite recycling of Plutonium with “standard” MOX

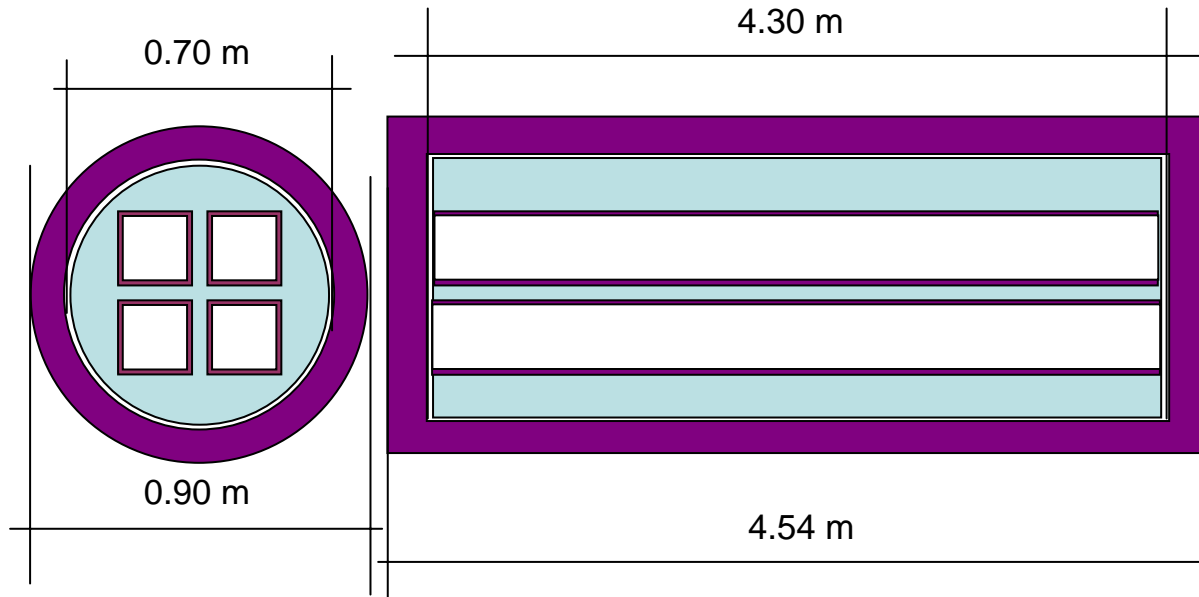
Three long-term scenarios :

- B1. Gen IV scenario: infinite recycling of Plutonium and Minor actinides in fast reactors.
- B2. Simplified double strata: LWR + ADS.
- B3. Double strata + Fast reactors + ADS.

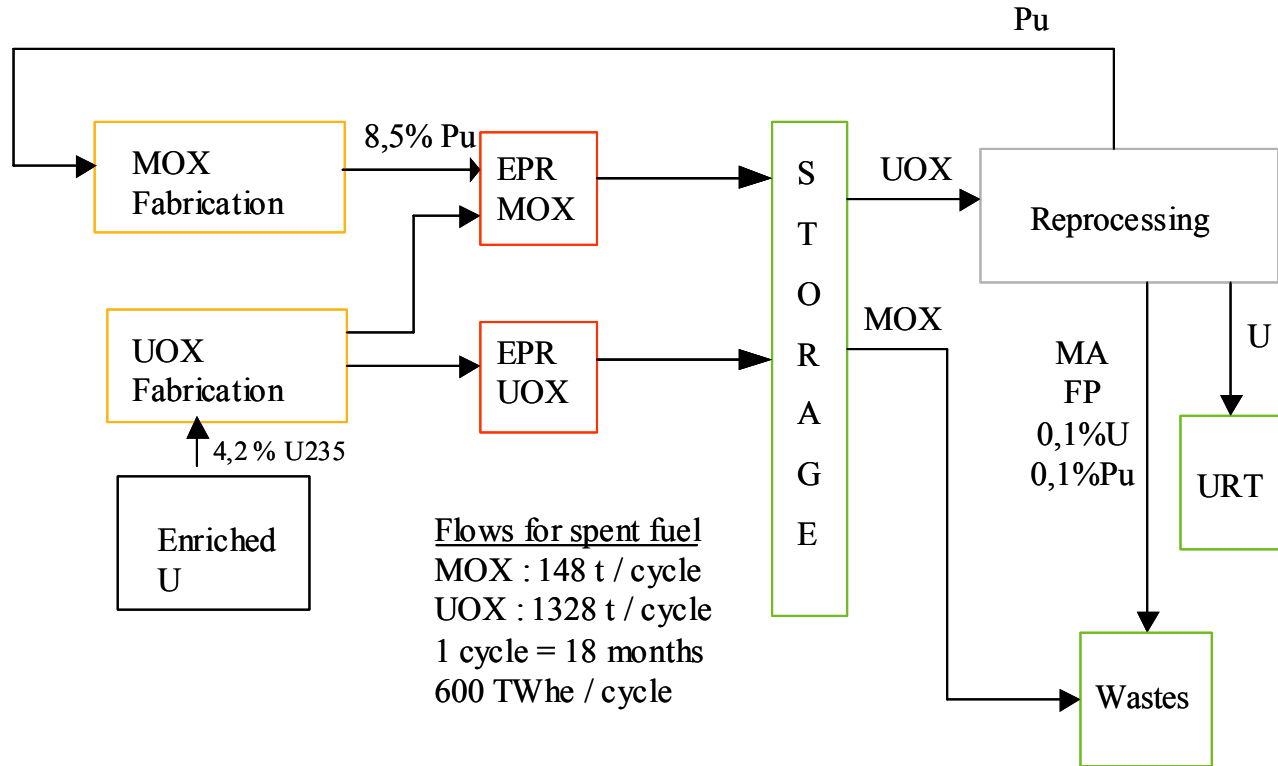
Scenario A1 - Reference



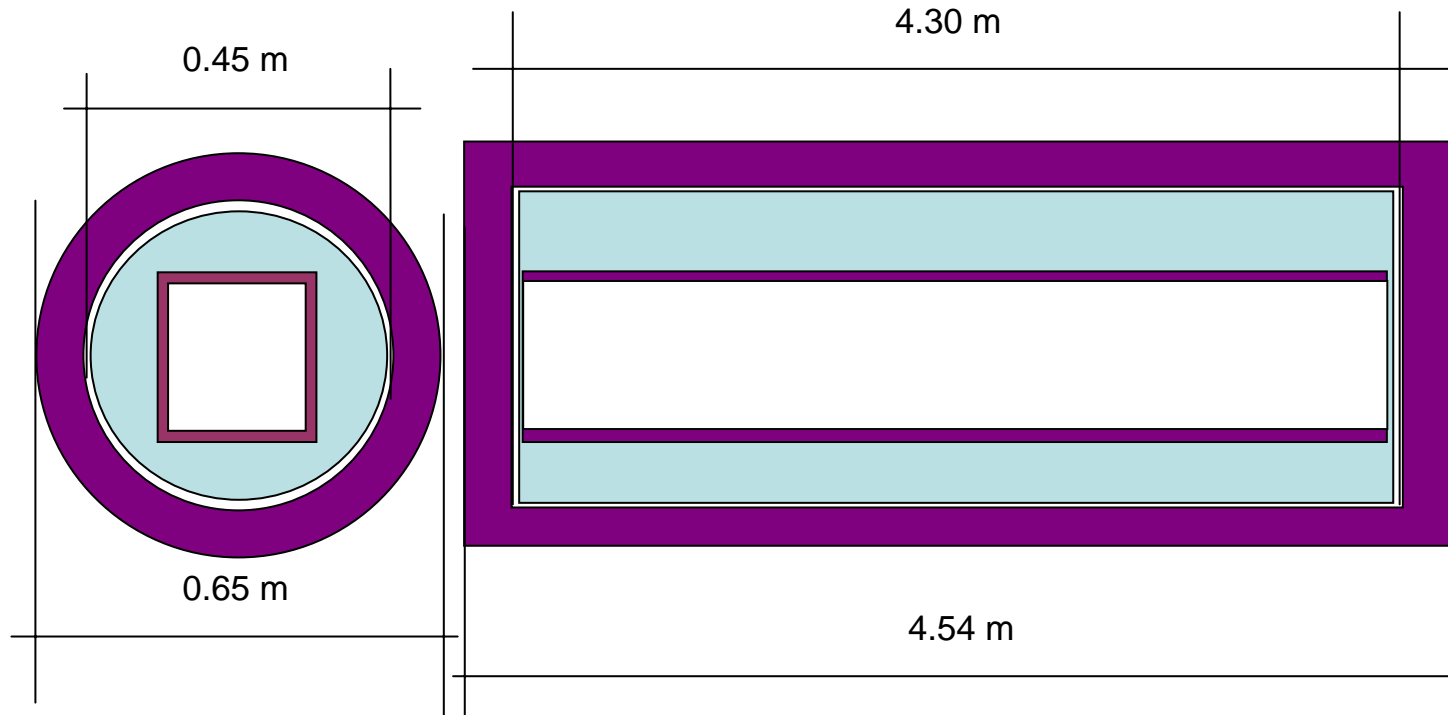
Scenario A1: Waste Package for 4 UOX spent fuel assemblies



Scenario A2: mono-recycling of plutonium in LWRs

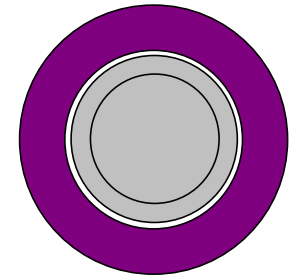
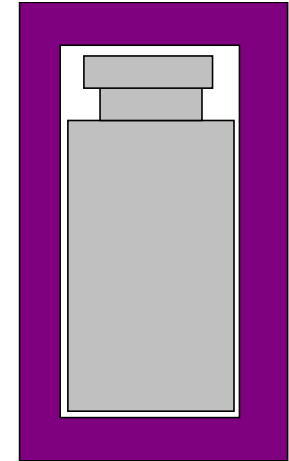


Scenario A2: Waste Package for 1 MOX spent fuel assembly

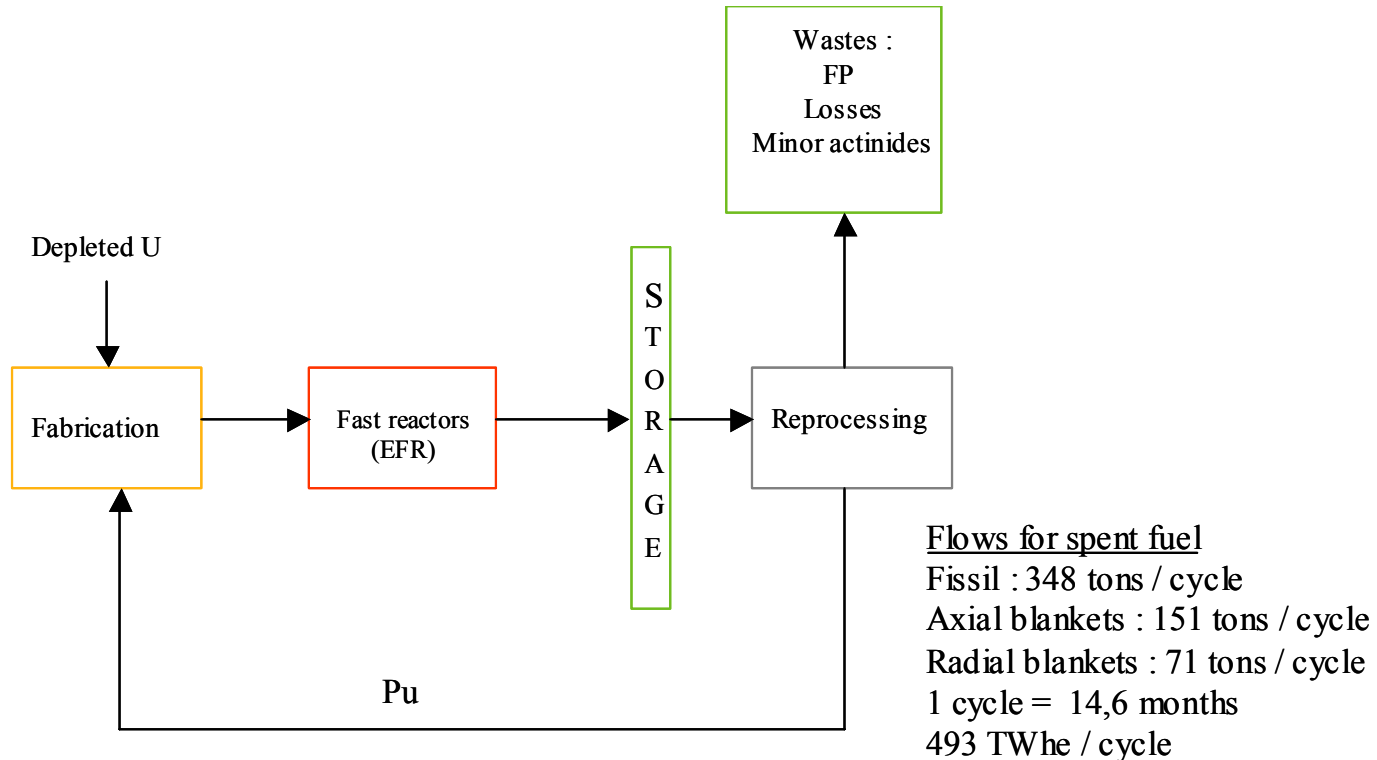


Universal Canister (scenarios A2, A3, B1, B2 and B3)

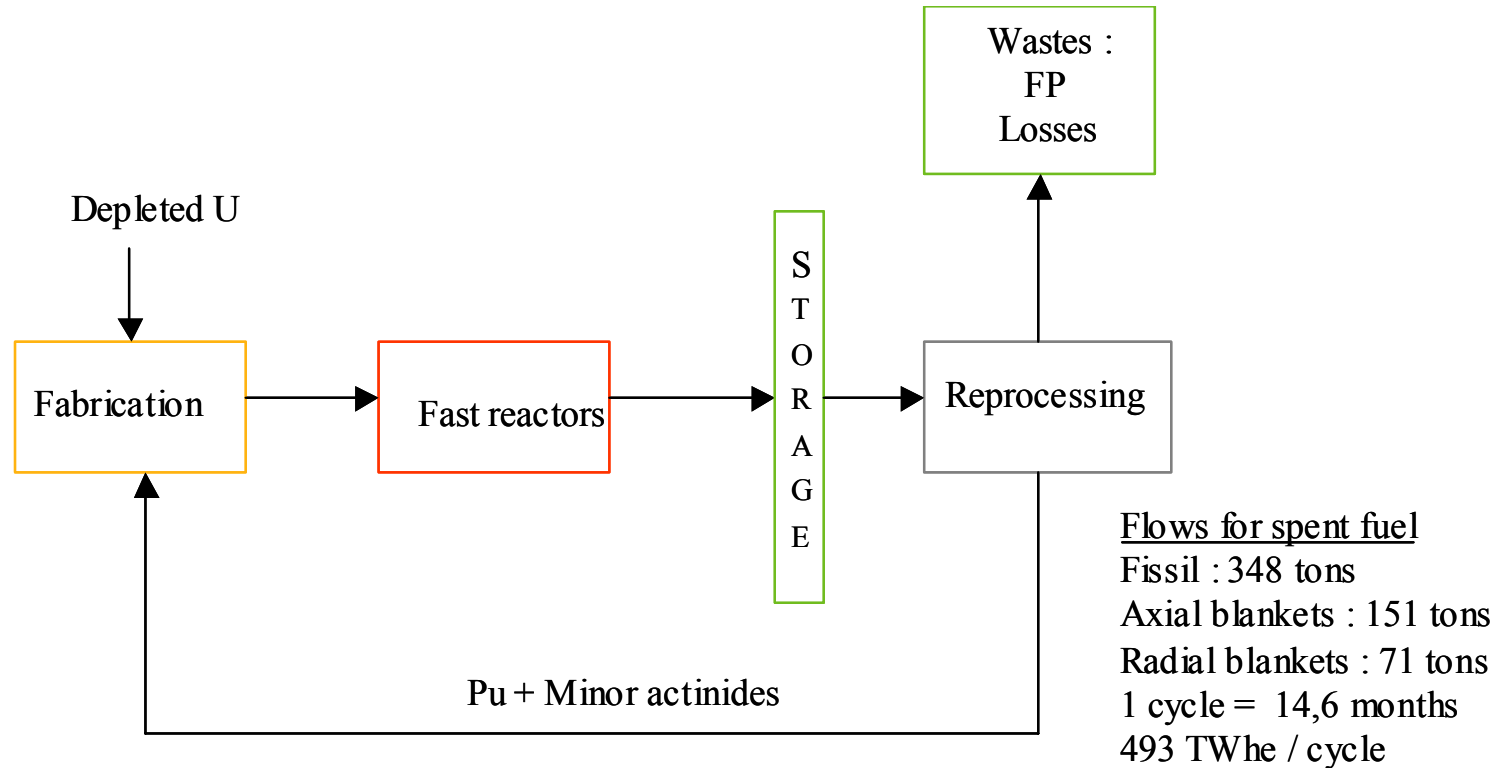
| | |
|----------------------|---|
| Material | Stainless Steel (C: 0.15%; Cr: 24%; Ni: 13%) |
| Physical dimensions: | |
| - Length | 1 338 mm |
| - External Diameter | 430 mm |
| - Wall thickness | 5 mm |
| Mass: | |
| - Total | 492 Kg |
| - Empty | 80 Kg |
| Volume: | |
| - External | 175 l |
| - Internal | 170 l |
| - Vitrified Waste | 150 l |



A3: "Industrial" scenario, infinite recycling of Plutonium with "standard" MOX.

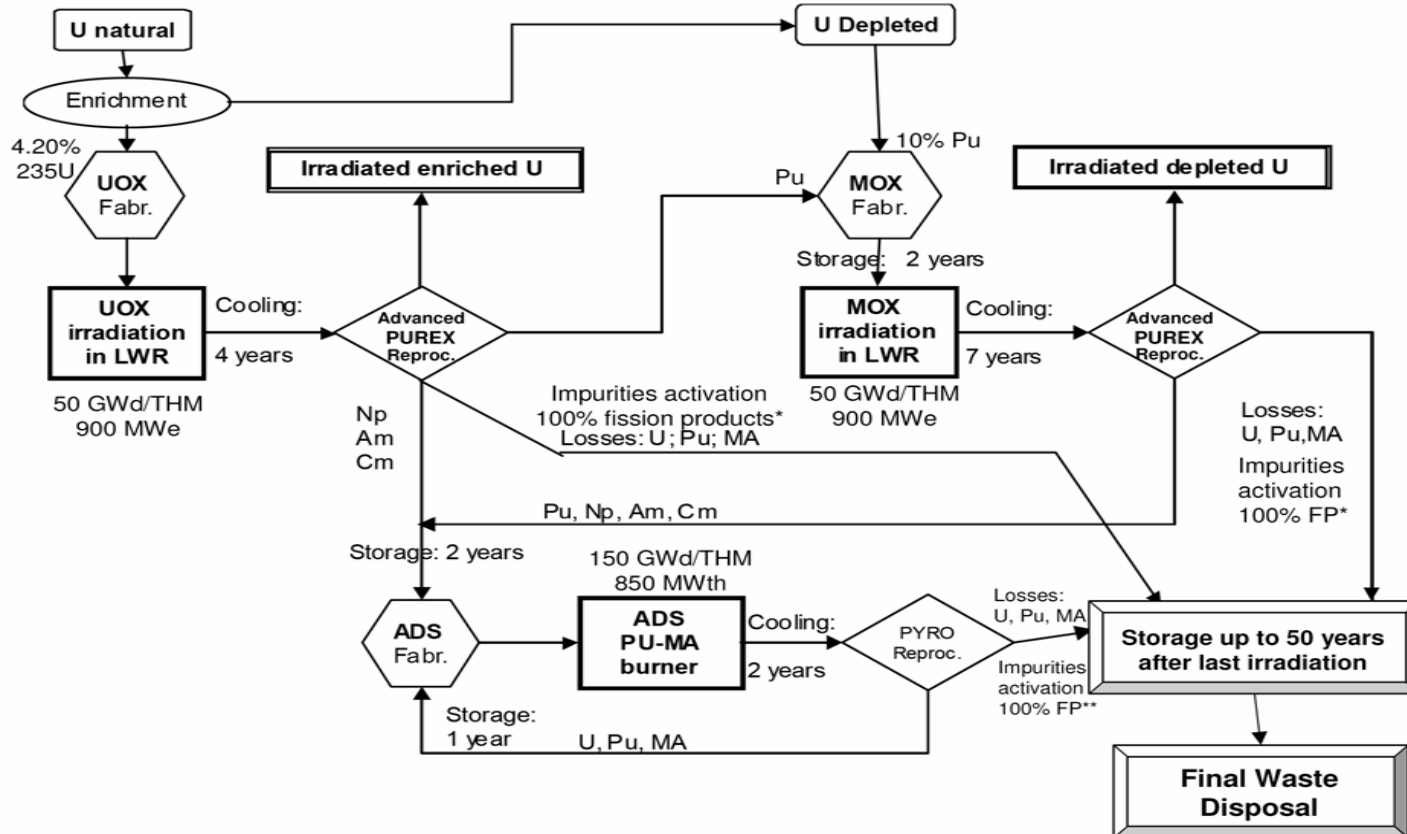


Scenario B1: Fast Neutron – Gen IV scenario



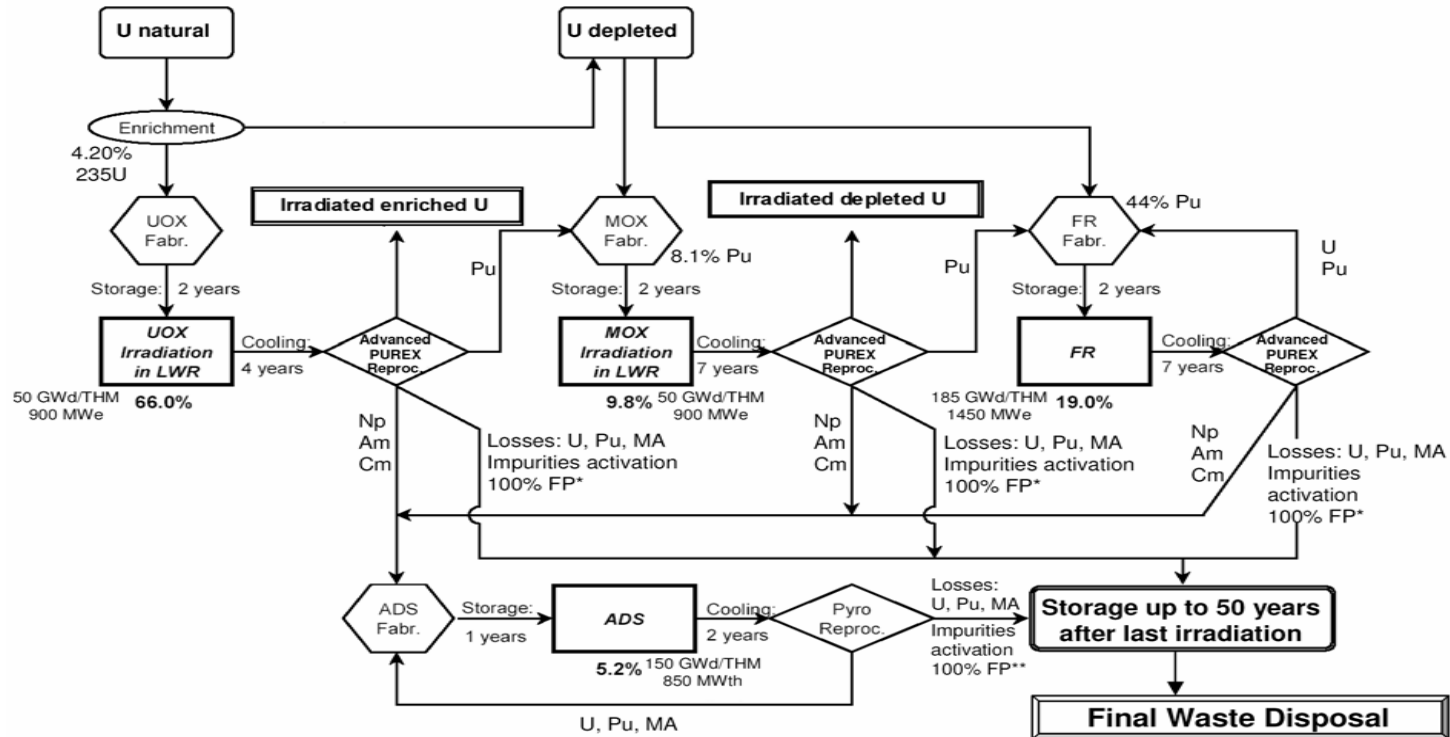
Scenario B2 : Simplified “double strata” scenario with LWR and ADS

RED IMPACT Scenario B2: Simplified double strata scenario with LWR and ADS



B3 – Double Strata Scenario with LWRs, FRs, ADS

RED IMPACT Scenario B3: Double strata scenario with fast reactor (LWR, FR and ADS)



Maturity of technologies

| Scenario | Description | Technologies needed (excepted final repository) | Year of availability |
|----------|--|---|----------------------|
| A1 | Once through fuel cycle in Gen II & III reactors | None | Available |
| A2 | Mono-recycling of Pu in Gen III reactors | None | Available |
| A3 | Multi recycling of Pu in Sodium Fast reactor | Fabrication and reprocessing of MOX fuel for FNR | 2020 |
| B1 | Mono recycling of Pu in Gen III reactor + Burning of Pu and MA in ADS | Partitioning ADS transmuter and associated fuel cycle | 2050 (QG) |
| B2 | Mono-recycling of Pu in Gen III reactor + Multi recycling of Pu in Gen IV Reactor + Burning of MA in ADS | Partitioning FNR and associated fuel cycle ADS transmuter and associated fuel cycle | 2050 (QG) |
| B3 | Multi recycling of U, Pu and MA in Gen IV reactors | FNR and associated fuel cycle (including MA) | 2050 (QG) |

Considered hydro-metallurgical processes, their goal and the state of the art

| Objective | Process | Status |
|--|---|--------------------------|
| Separation of U, Pu, FP+MA | PUREX | Industrial-scale process |
| Separation of U, Pu+FP+MA | UREX | Industrial feasibility |
| MA partitioning, one-extraction-cycle process | DIDPA process, SETFICS, PALADIN | Scientific feasibility |
| An+Ln co-extraction | TRUEX, DIAMEX, TRPO | Technical feasibility |
| An, Ln separation | TALSPEAK, CTH, SANEX, CYANEX, ALINA, BTP | Technical feasibility |
| Am, Cm separation | SESAME, Am precipitation | Technical feasibility |
| I, Np, Tc recovery | Advanced PUREX | Industrial feasibility |
| Cs and/or Sr recovery | Calixarenes, titanilic acid | Technical feasibility |

Pyroprocessing (necessary mainly for ADS recirculation schemes)

- Involve several techniques such as :
volatilization, liquid-liquid extraction using non-miscible metal-metal phases or metal-salt phases, electro-refining in molten salt, fractional crystallization, etc.
- Lack of reliable technical data. Assumptions and qualified guesses needed!

Reprocessing assumptions

| | | SCENARIO | | | | | | | | | | | |
|--|---|--------------|--------------|--------------|------------------------------|------------------------------|--------------|--------------|--------------|--------------|--------------|--------------|--------------|
| | | A2 | A2.b | | A3 | B1 | B2 | | | B3 | | | |
| REPROCESSING TYPE | | Stand. PUREX | Stand. PUREX | Stand. PUREX | Stand. PUREX, | PYRO | Adv. PUREX | Adv. PUREX | PYRO | Adv. PUREX | Adv. PUREX | PYRO | PYRO |
| REPROCESSED FUEL | | UOX (PWR) | UOX (PWR) | MOX (PWR) | Blankets + Fissile Core (FR) | Blankets + Fissile Core (FR) | UOX (PWR) | MOX (PWR) | ADS | UOX (PWR) | MOX (PWR) | MOX (FR) | ADS |
| ACTINIDES to HLW | U | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 |
| | Pu | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 |
| | MA | 1 | 1 | 1 | 1 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 |
| FISSION PRODUCTS to HLW | Noble Gases | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 |
| | Iodine | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 | 0.001 |
| | Noble Metals | 1 | 1 | 1 | 1 | 0 | 1 | 1 | 0 | 1 | 1 | 0 | 0 |
| | Others, including volatile (Cs, Br) and partially soluble (Mo, Tc and Sb) | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 |
| ACTIVATION PRODUCTS (Fuel Impurities) to HLW | All, including volatile (H, C, N, F and Cl), etc. | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 |
| Zr and N (ADS fuel matrix) to HLW | | - | - | - | - | - | - | - | 0 | - | - | - | 0 |
| MASS of HLW to Universal Canister | | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. | 40 Kg +Impu. |

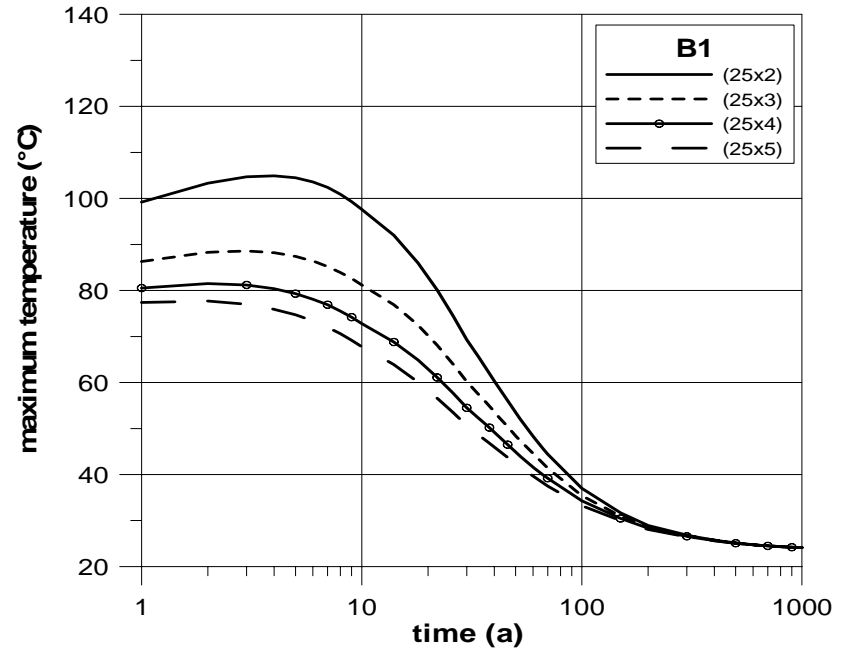
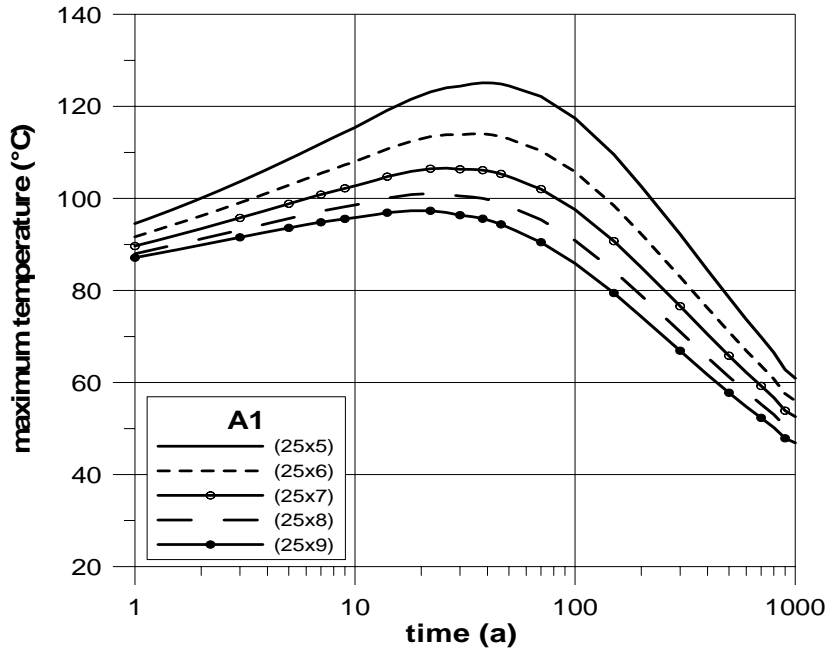
3 Different Geological Disposal Models

- Granite (e.g. Swedish, Spanish, Czech models)
- Clay (Belgium)
- Rock Salt Formation (Germany)

Samples of results:

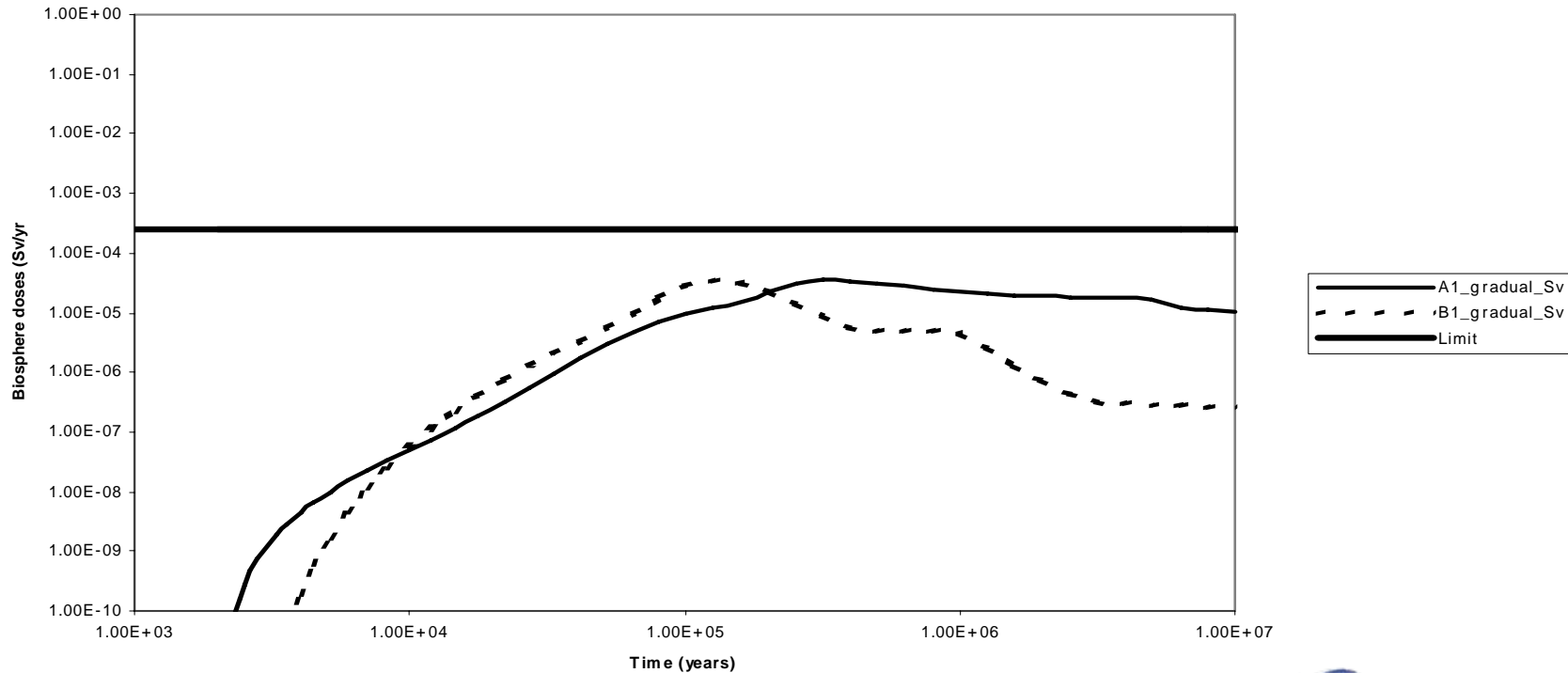
Granite:

Evolution of maximal temperature at the surface of the canisters calculated for scenarios A1 and B1



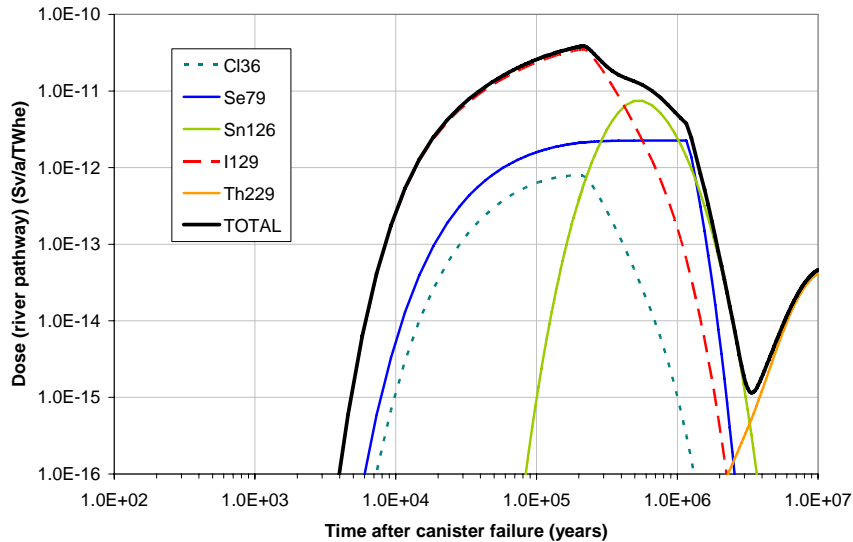
Granite

*Comparison of biosphere doses from A1 and B1 scenarios
(instant failure of canisters)*

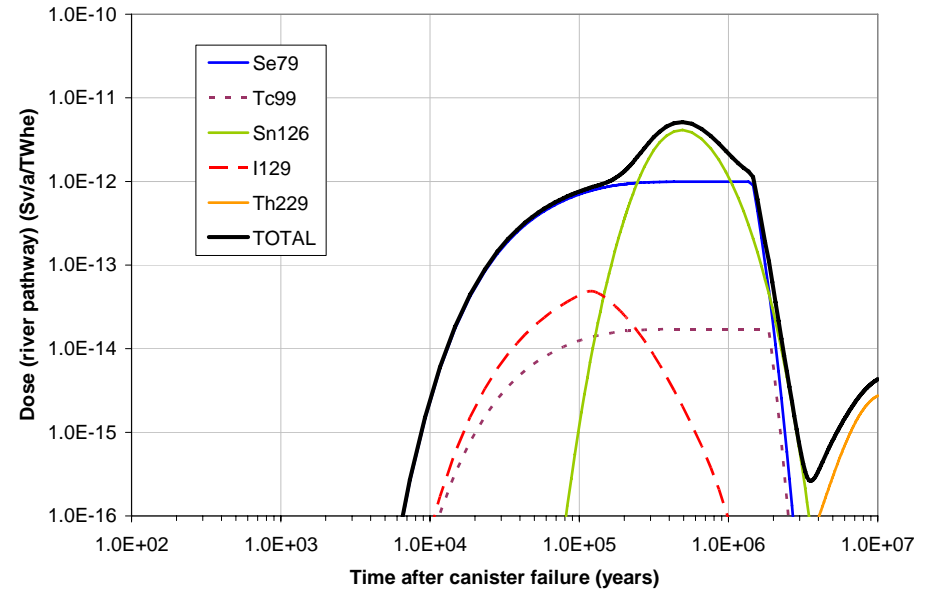


Clay: Dose via the river pathway per TWh_e

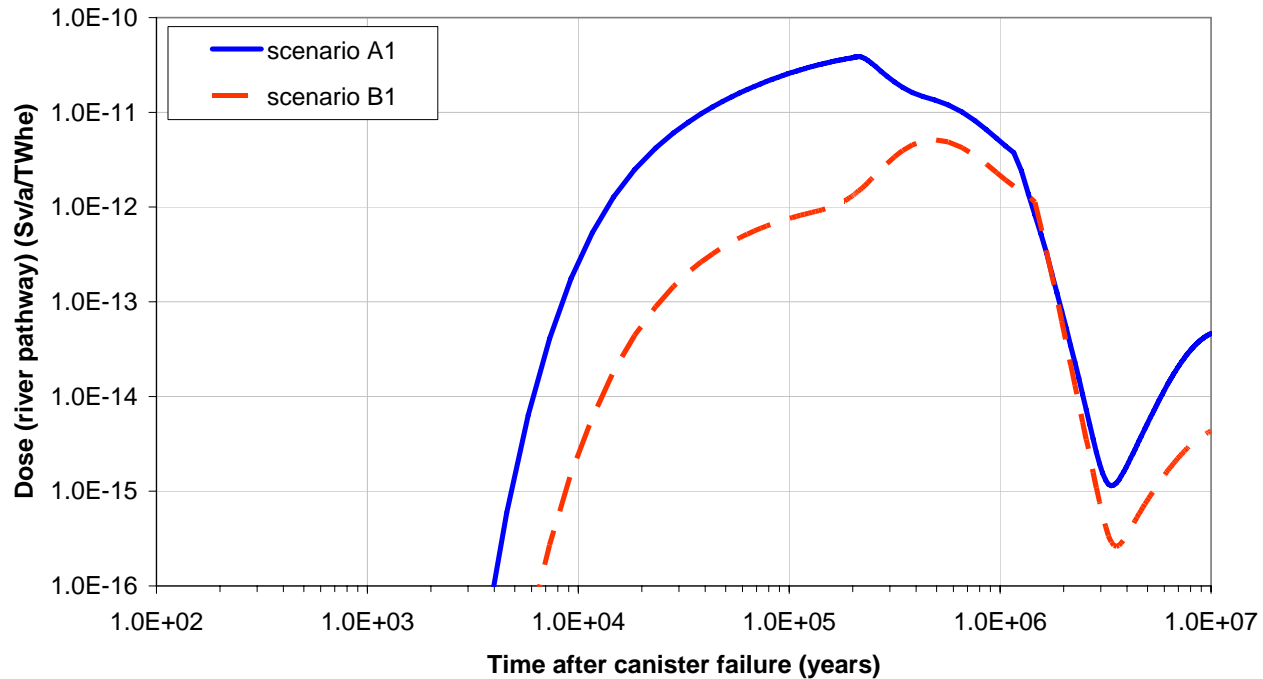
A1



B1

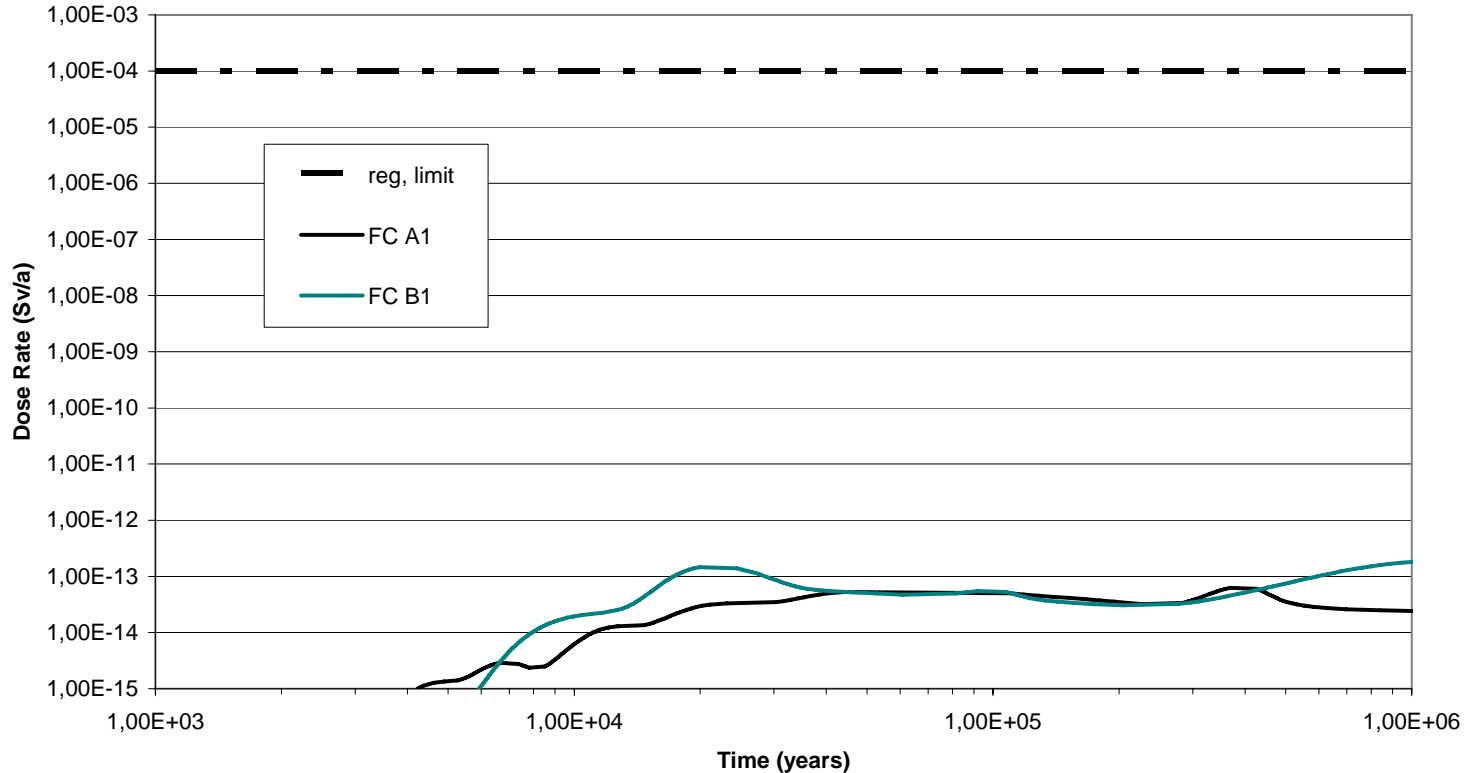


Clay: Total doses via the river pathway per TWh(e) calculated for scenarios A1 and B1



Salt: dose rate for possible releases coming from waste of fuel cycle A1 and B1

Calculated Dose Rate
fuel cycles A1 and B1

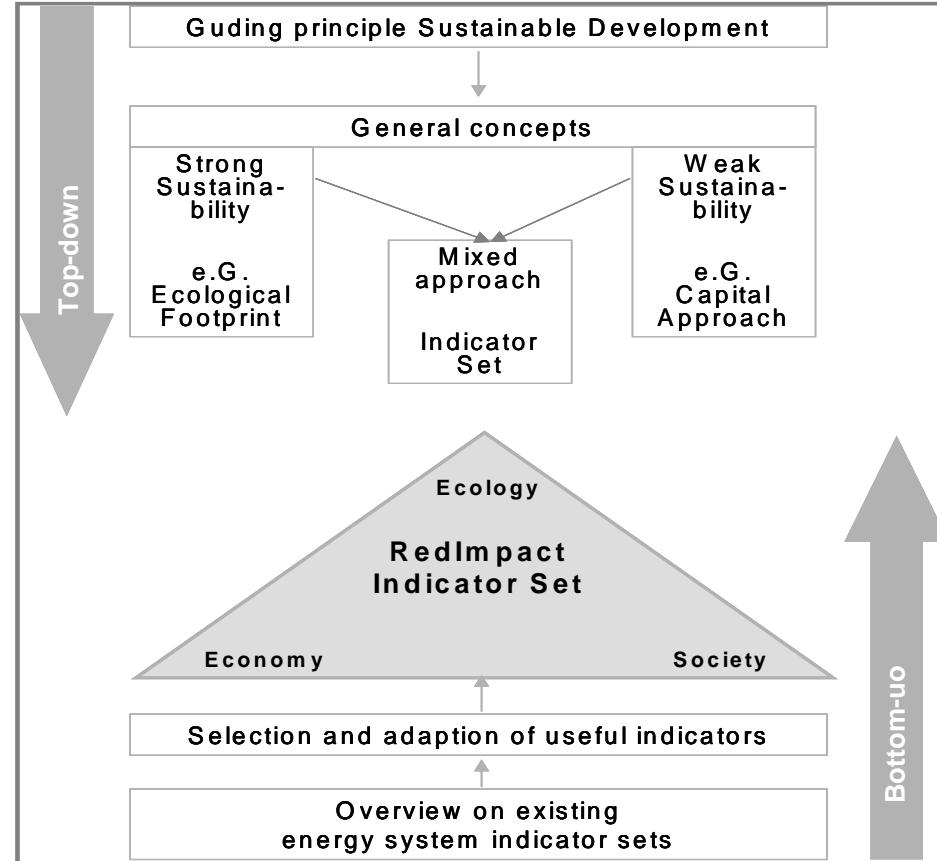


Performance indicators

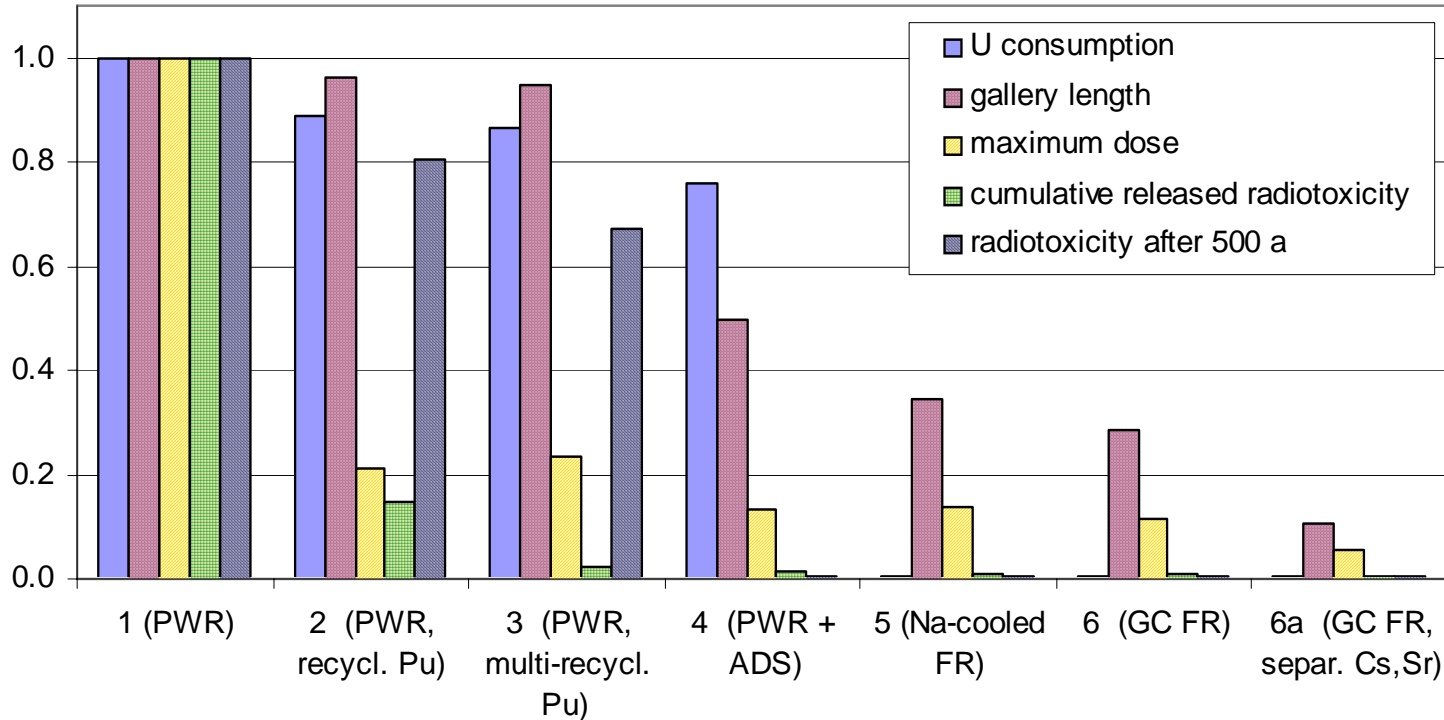
The indicators have been divided into two major groups:

- “Technical” indicators partitioned into three groups:
 - indicators based on the composition of the waste;
 - indicators related to the size of the repository;
 - indicators related to the long-term performance of the repository system:
 - individual annual dose;
 - radiotoxicity flux released into the biosphere;
 - integrated radiotoxicity flux released into the biosphere.
- Economic, environmental and societal/sustainability (EES) indicators

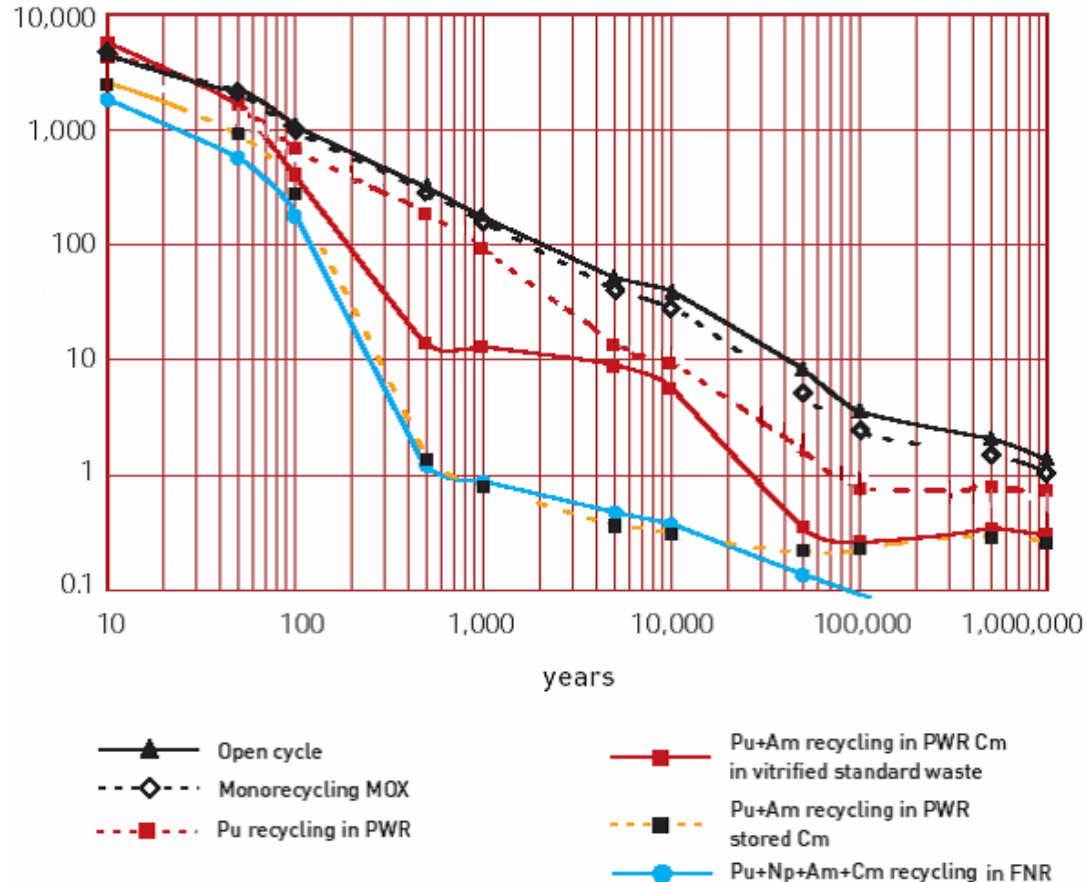
Sustainability Indicators Identification



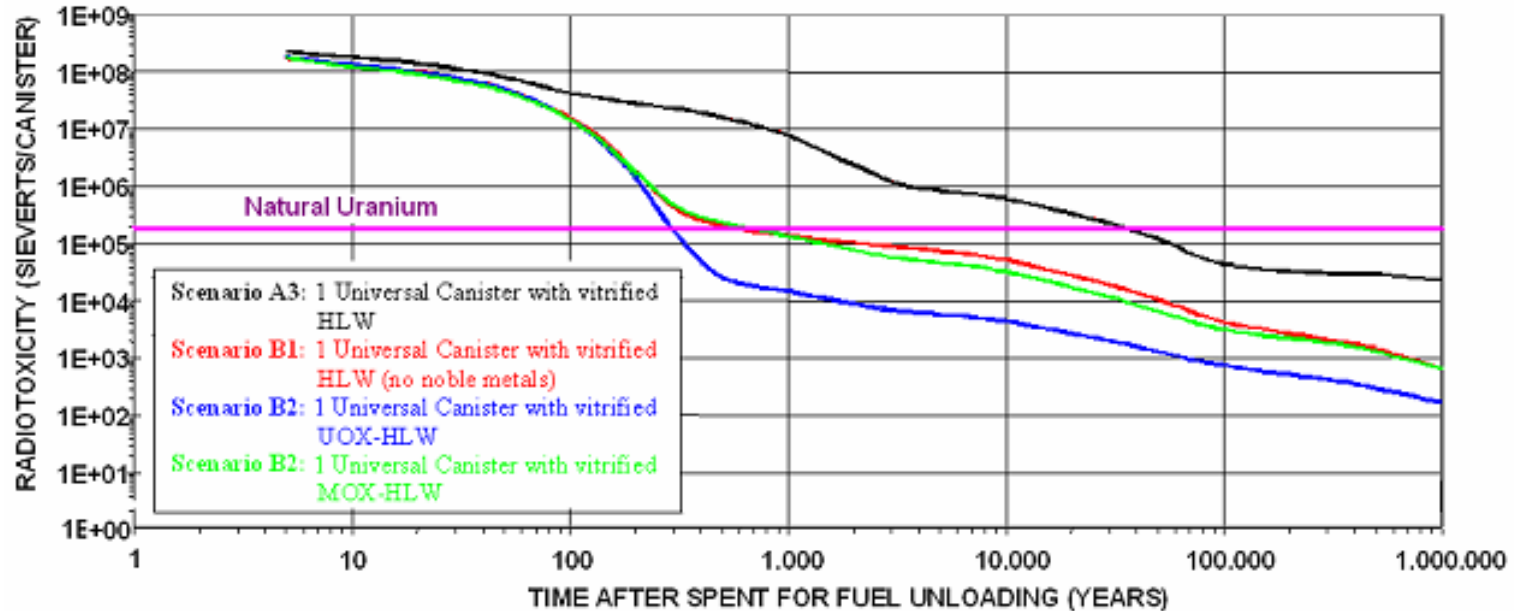
Comparison of different fuel cycle strategies in terms of radiological and disposal aspects



Decay heat per TWhe as a function of time in a repository for different fuel cycles



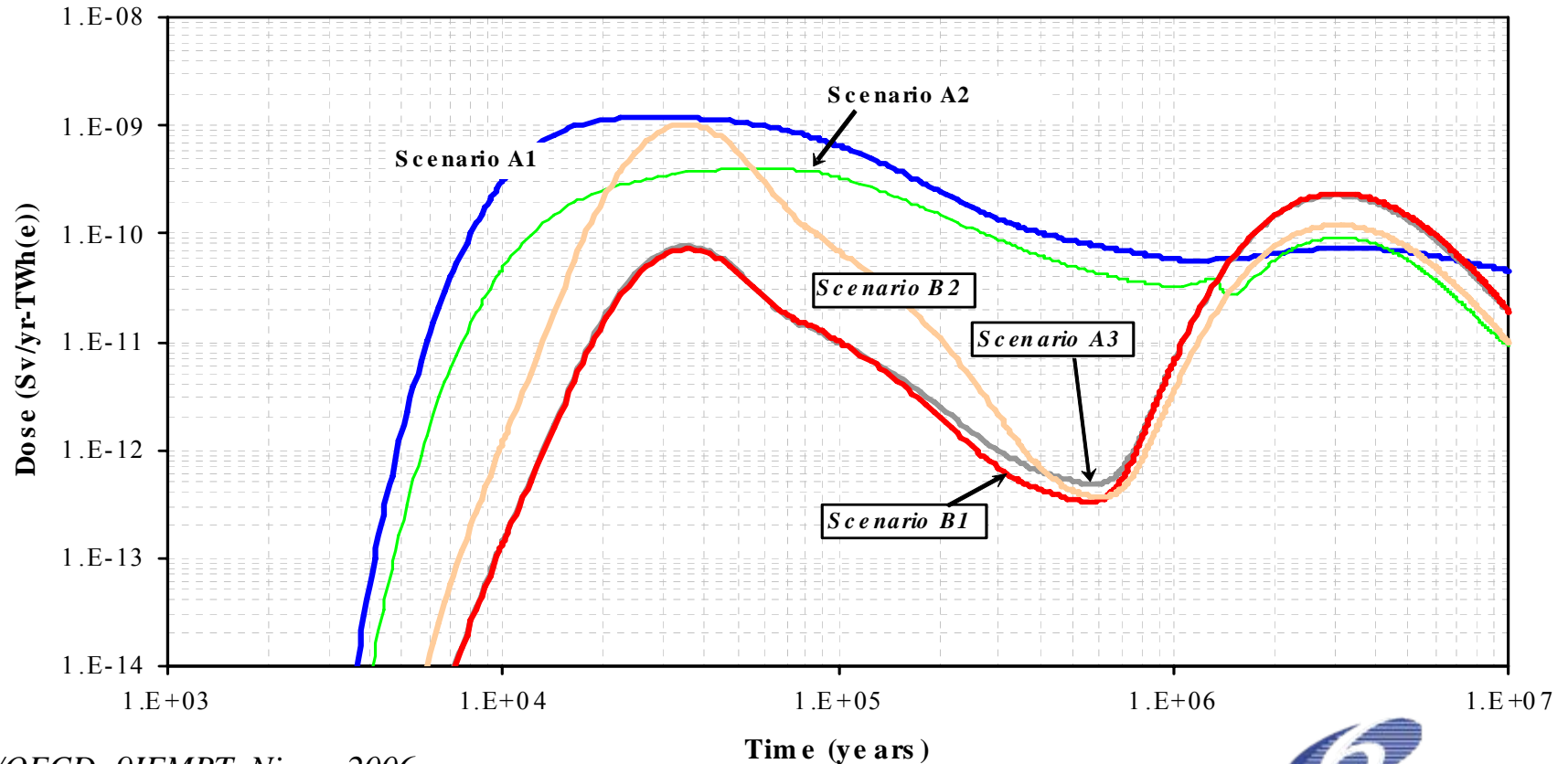
Ingestion radiotoxicity after disposal for different scenarios



Impact of fuel cycle on waste quality for disposal

| | Scenarios | | | | | | | |
|--|------------|-------------|-------|------------|------------|-------------|-----------|-----------|
| | A1 | A2 | | A3 | B1 | B2 | | |
| Waste form | UOX | MOX | Glass | Glass | Glass | UOX-Glass | MOX-Glass | ADS-Glass |
| Waste package external length (m) | 4.54 | 4.54 | 1.6 | 1.6 | 1.6 | 1.6 | 1.6 | 1.6 |
| Waste Volume ($m^3/TWh(e)$) | 3.8 | 0.8 | 1.3 | 1.3 | 1.2 | 1.06 | 0.13 | 2.2 |
| Waste Volume per scenario ($m^3/TWh(e)$) | 3.8 | 2.14 | | 1.3 | 1.2 | 3.44 | | |
| Waste packages/TWh | 1.33 | 0.54 | 2.5 | 2.4 | 2.3 | 2 | 0.24 | 4.2 |
| Thermal power at disposal time (W/package) | 1564 | 1549 | 486 | 529 | 314 | 420 | 331 | 31 |

All scenarios – Radiological Impact: Normalised Annual Dose (Sv/a TWh_e)



RedImpact Final Goal: Acceptance and Decisions through understanding

- The RED-IMPACT project follows a multi-disciplinary approach by federating different technical and scientific areas such as specialists from **reactor technology, fuel cycle evaluation, reprocessing (partitioning), waste management, disposal issues, economical assessment and social science** with regard to public and market acceptance aspects. P&T is not understood only as a very future option but as a strategy which can be stepwise implemented already starting:
 - in the short term by making use of existing reactors and fuel cycle facilities,
 - via more advanced reactor systems (Gen III & Gen IV), fuels, reprocessing & conditioning technologies, in the mid term,
 - towards very ambitious special waste transmuter systems like ADS together with the related partitioning techniques, in the long term.

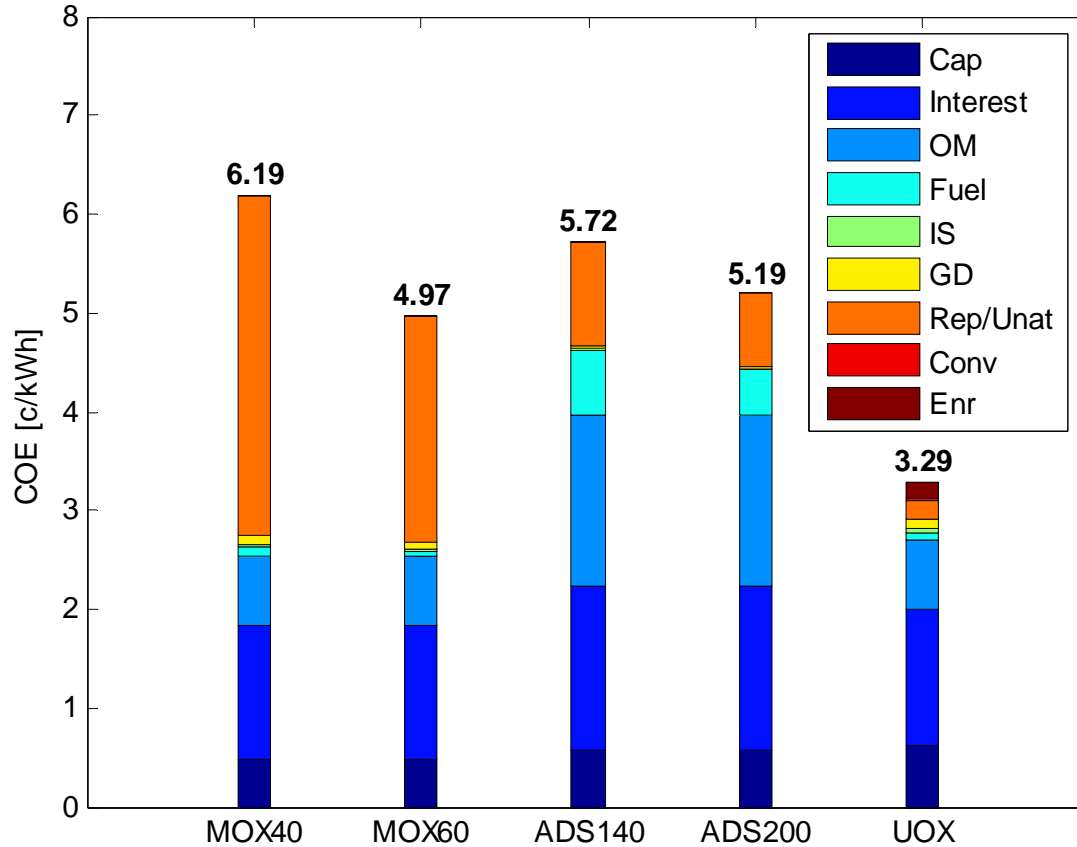
RedImpact Final Goal: Acceptance and Decisions through understanding

- Fuel cycle scenarios with their boundary conditions have been well defined. Methodology and assessment tools have been agreed and **well benchmarked**. Most of the calculations are in the final stage and an interim report summarizing the first results has been prepared.
- The project provides recommendations for the implementation of pragmatic P&T strategies under different political environments following general sustainability criteria to improve public acceptance

Added on values 1:

- Subtask: Nuclear Spent Fuel Management Scenarios for Sweden:
 - Phase out with direct disposal
 - Burning plutonium and minor actinides as MOX in BWR
 - Burning plutonium and minor actinides as MOX in PWR
 - Burning plutonium and minor actinides in ADS
 - Combined LWR-MOX plus ADS

Cost of electricity comparison for various scenarios with 40, 60, 140, and 200 GWd/tHM of burnup



Added on values 2:

- New Fuel Cycle Analysis and OPTIMIZATION (RED-IMPACT) code under development at KTH:
 - Best possible physics – generation of dedicated one-group data libraries for each specific case
 - Matrix method
 - Simulation of the whole fuel cycle from FRONT-FRONT to Back-End including legacy of existing waste
 - Regional solutions as an option
 - Economical calculation
 - Uncertainty analysis
 - Very advanced graphical interface