

PAUL SCHERRER INSTITUT



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# Decommissioning of Nuclear Facilities in Switzerland – Lessons learned

HRP/IAEA/NEA Decommissioning workshop – February 7, 2017

← Basel

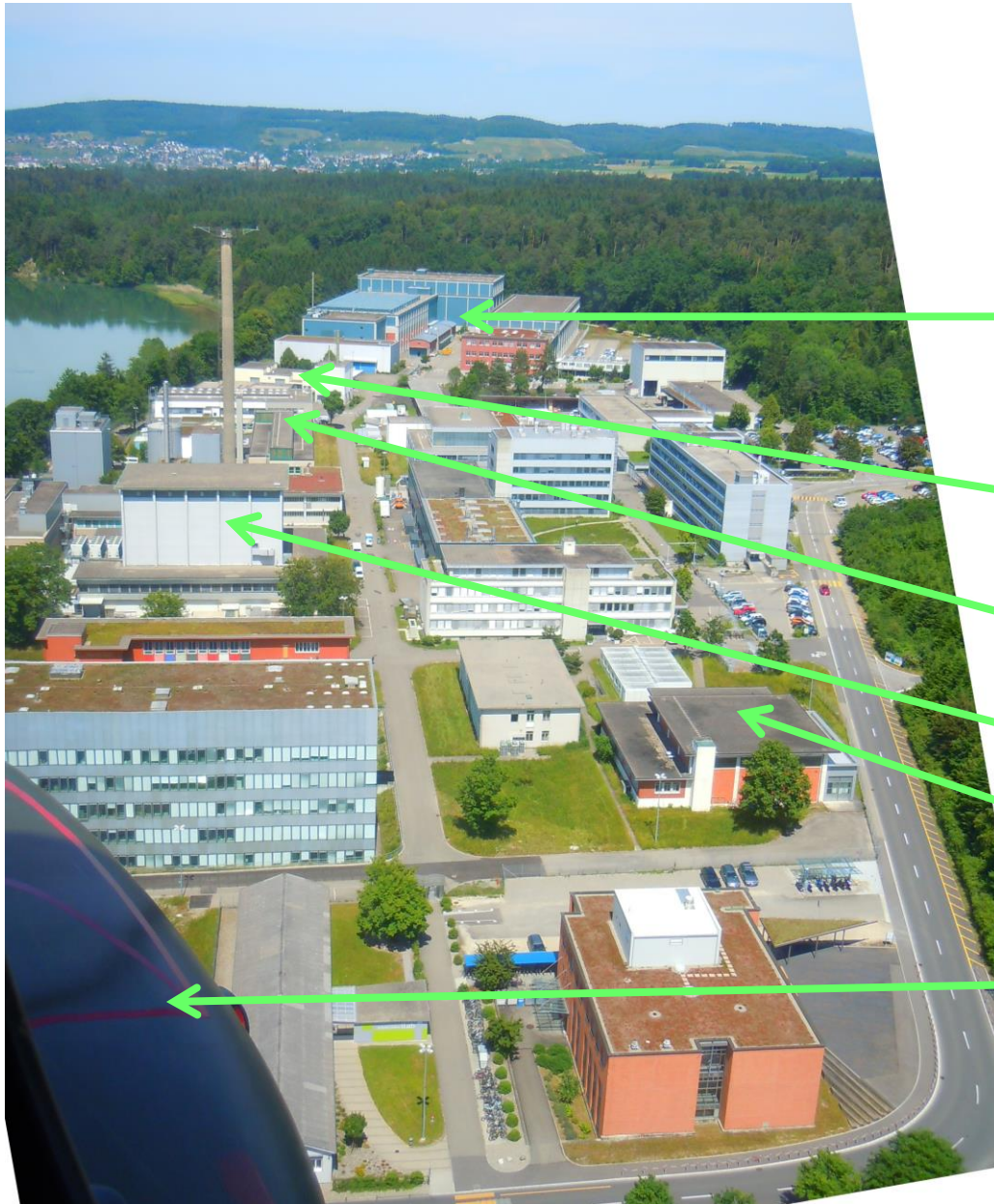
Germany ↑

Aarau/Bern ↓

Zürich →



# Nuclear installations on the PSI area



- (ZWILAG)
- AERA  
with VVA\*
- Hotlabor
- DIORIT\*
- SAPHIR\*
- PROTEUS\*

\*Post-operation phase/  
Decomm./Dismantling

First reactor in Switzerland; used for isotope production, reactor training, neutron source for various experiments

1955 USA exposed a reactor at the “Atoms for Peace” conference in Geneva

1956 Laying of the cornerstone in Würenlingen

1957 First criticality

1960 Approval by Swiss government

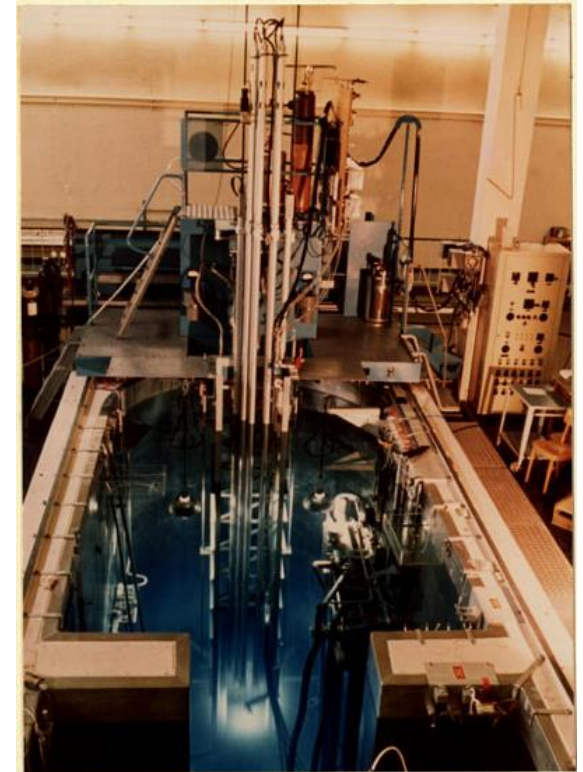
1985 Approval for 10 MW

1993 Final shutdown

2000 Decommissioning ordinance

2008 Dismantling of the pool completed

2015 Cleanout of the KBL (“Kernbrennstofflager”)





ENSI-Inspection at 7. of April, 2016

Proprietary Swiss development. Goal was the construction of industrial applicable reactors for material testings and experiments.

1960 Operation with natural uranium and D<sub>2</sub>O as coolant and moderator.

1966 Upgrading from 20 MW to 30 MW.

1972 (after modification): Operation with LEU.

1977 Final shutdown.

1982 Partial dismantling; continued 1988-1993.

1994 Approval of dismantling the reactor.

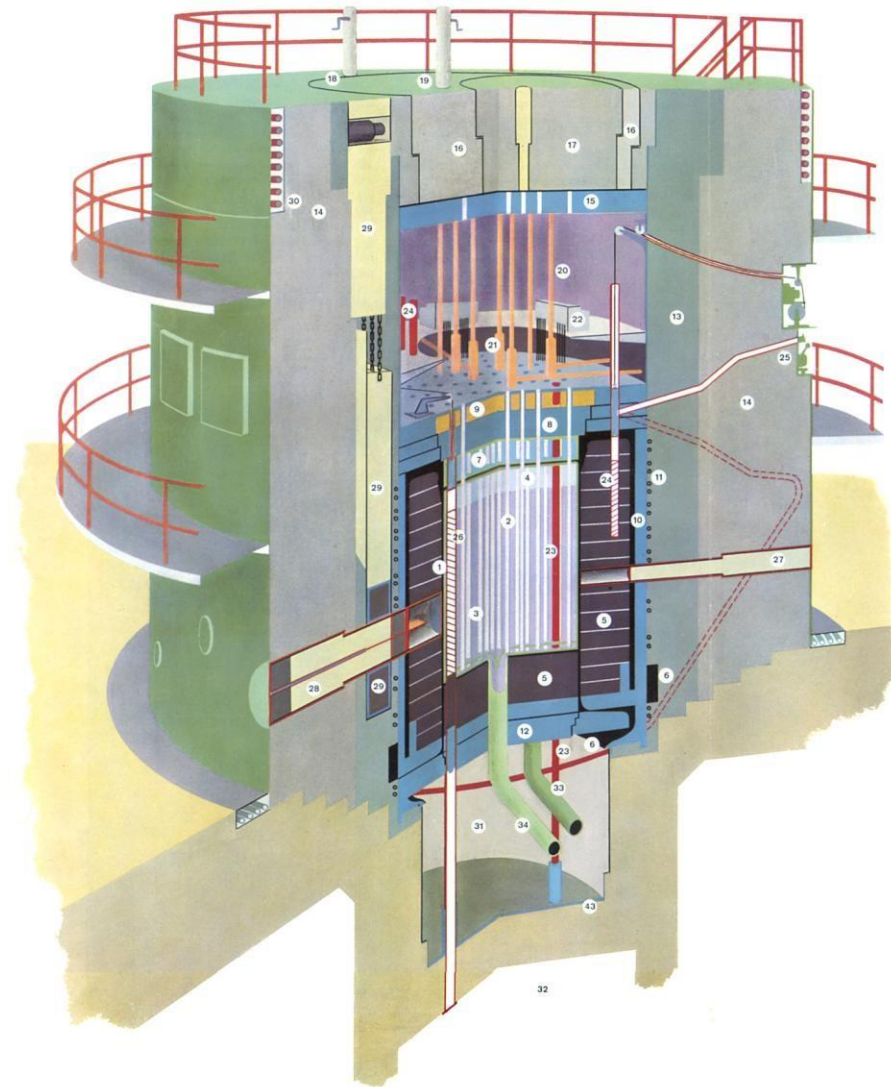
2005 Asbestos was found → interruption until 2009.

2013 Dismantling of biological shielding

2016 Cutting of the „Arbeitsboden“ (22 t activated Fe)

2019 (?) 2. Decommissioning ordinance for greenfield







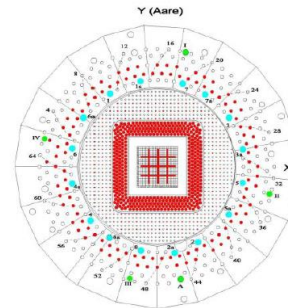
March 2012



September 2012

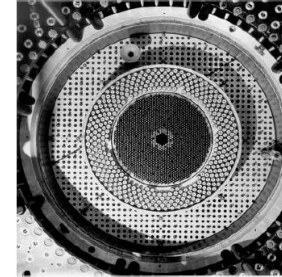


HPLWR-today

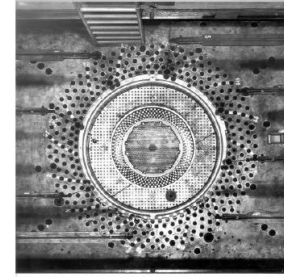


Zero power reactor for different experiments with various fuel arrangements (U, MOX).

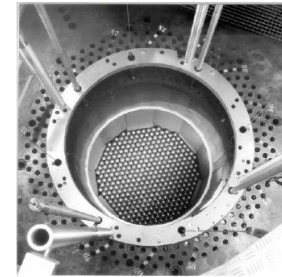
GCFR-70s



HCLWR-80s



HTR-90s



LWR-00s



GCFR: gas-cooled fast reactor

HCLWR: tight-pitch, high conversion, light water reactor

HTR: modular high temperature reactor

HPLWR: high performance light water reactor fuel

2011 Final shutdown

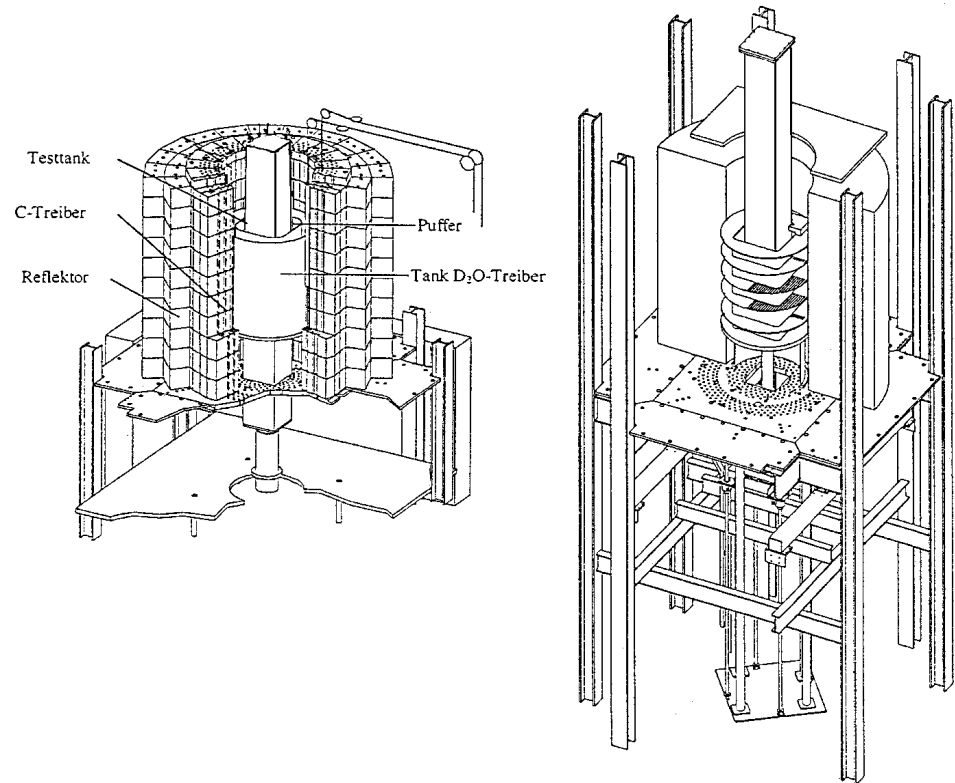
2012 Removal of fuel

2013 Application for decommissioning

2015 Approval of post-operation phase,  
deactivation of reactor instrumentation.

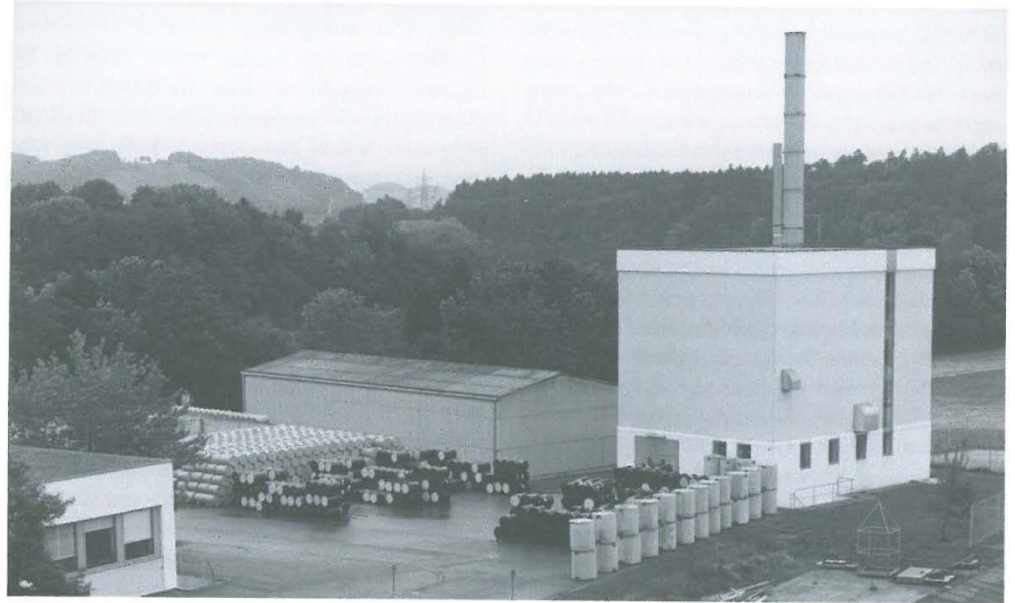
2016 Public obligation of the project. No  
objection.

2017 (?) Decommissioning ordinance



Part of the waste management facilities, „Anlagen zur Entsorgung radioaktiver Abfälle“ (AERA)

1974 – 2002 Operation of VVA



Incineration of solid low level waste

- Chemically more stable
- Volume reduction

Stabilization of the ashes with cement mortar in 200 l drums.

2011 Application for decommissioning

2012 ENSI expertise

2013 Public obligation

2014 Decommissioning ordinance

2015 Preparation of dismantling, building application

2016 Start of dismantling

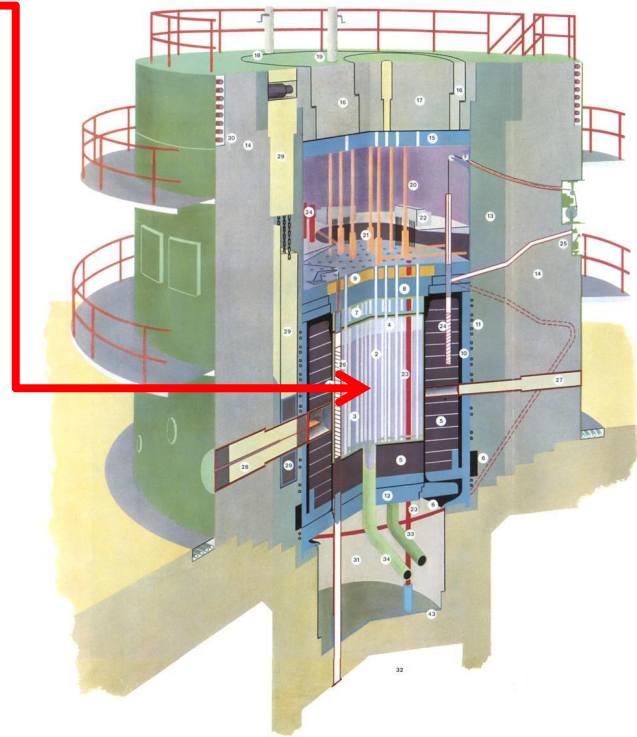


30.05.2016

## Main wastes generated:

- ***Aluminum***
- ***Graphite***
- Steel/Cast iron
- Concrete
- Asbestos

- A significant waste from decommissioning of nuclear facilities
- Contains mainly Co-60
- Two reactor tanks from DIORIT I and II, approx. 2 x 1.5 tons
- Dose rates up to 700 mSv/h
- Various elements from SAPHIR, approx. 1.2 tons
- Cavities from proton accelerator, approx. 6.4 tons



# Aluminium conditioning

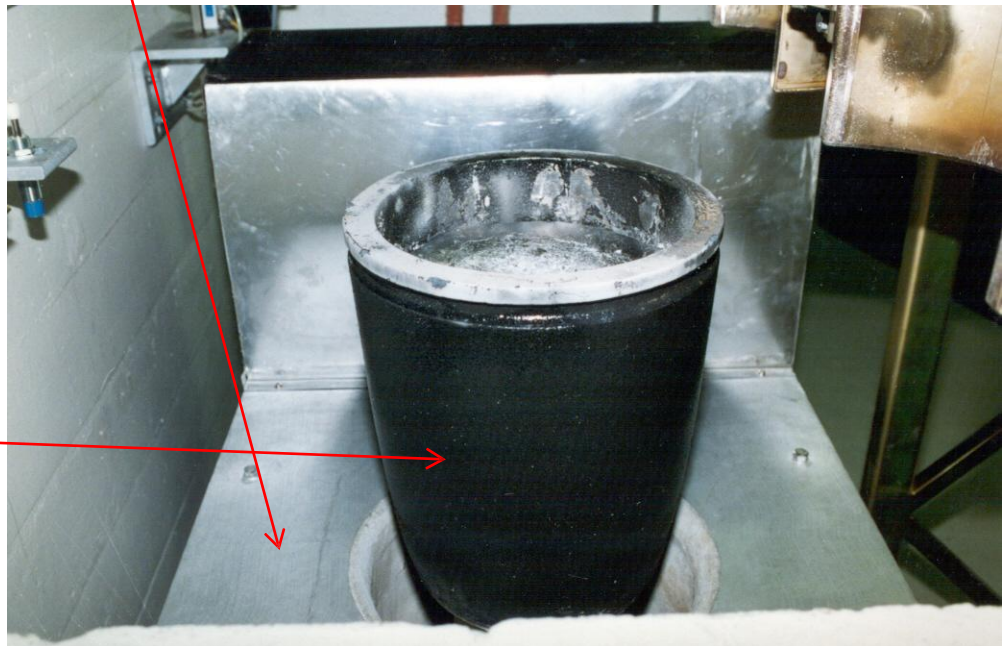


Al-basket

inductively  
coupled furnace

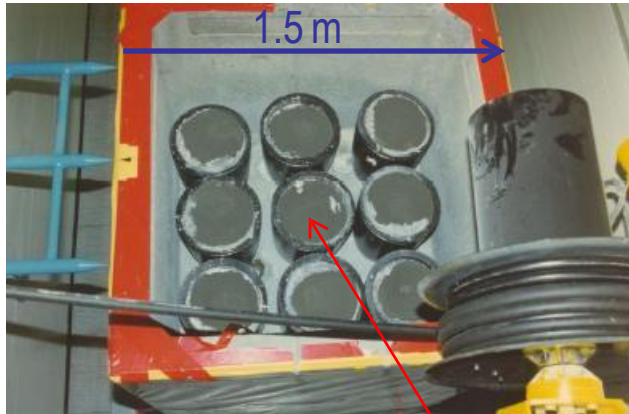
## Solution:

- Cutting of the Al-waste into pieces of 20 x 120 cm size
- Place the pieces in Al-baskets of 28 x 130 cm
- Melting the Al-baskets with the Al-waste in inductively coupled furnace into graphite/clay crucibles



graphite/clay crucibles

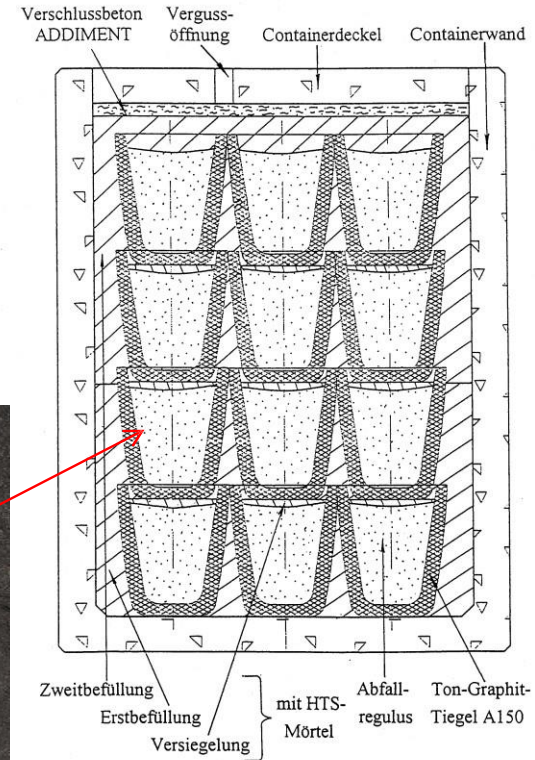
# Aluminium conditioning contd.



Crucibles filled with Al and placed in a PSI concrete container



36 mit Abfallreguli gefüllte Ton-Graphit-Tiegel in 16 t-Beton-Dünnwandcontainer Typ KC-T12 konditioniert



- Placing the crucibles with the melted Al-waste in a concrete container (KC-T12)
- Embedding the crucibles in PSI-mortar

H.-F. Beer, Complete Dismantling of the Research Reactor DIORIT, Strahlenschutzpraxis, 2013(2), p. 32-37



## **Result:**

- Reduced reactive surface of the Al in contact with the mortar with a slightly gas evolution before hardening of the mortar. After hardening there is no gas formation anymore!

# Graphite from DIORIT

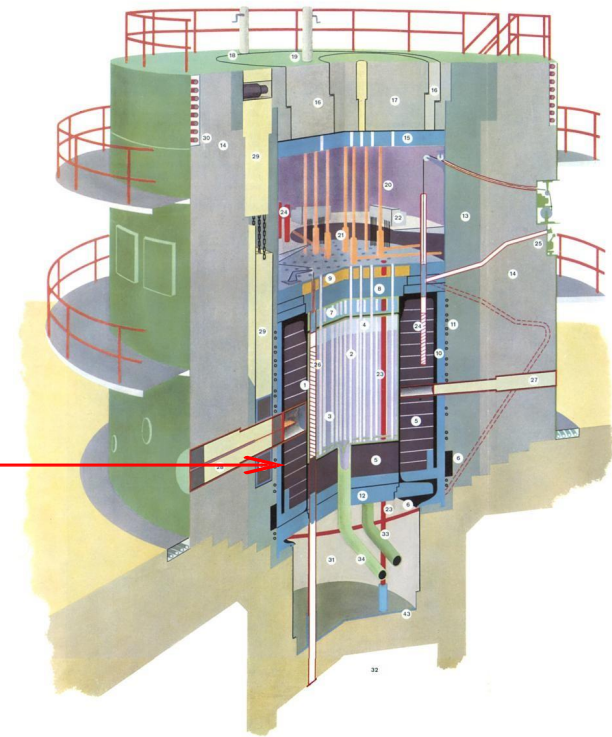
- A significant waste from decommissioning of nuclear facilities
- Contains long-lived nuclides C-14, Cl-36
- Stable mineral matrix (stable under geological conditions and high pH)

## Principles

- Minimise operational risks
- Minimise environmental impact
- Minimise costs

## Situation

- About 45 t activated graphite in the DIORIT
- in segments of about 50 kg
- Dose rate between 30 and 2000  $\mu\text{Sv/h}$



## Waste Container

### Grout:

Gap width 3 times of the maximum grain size

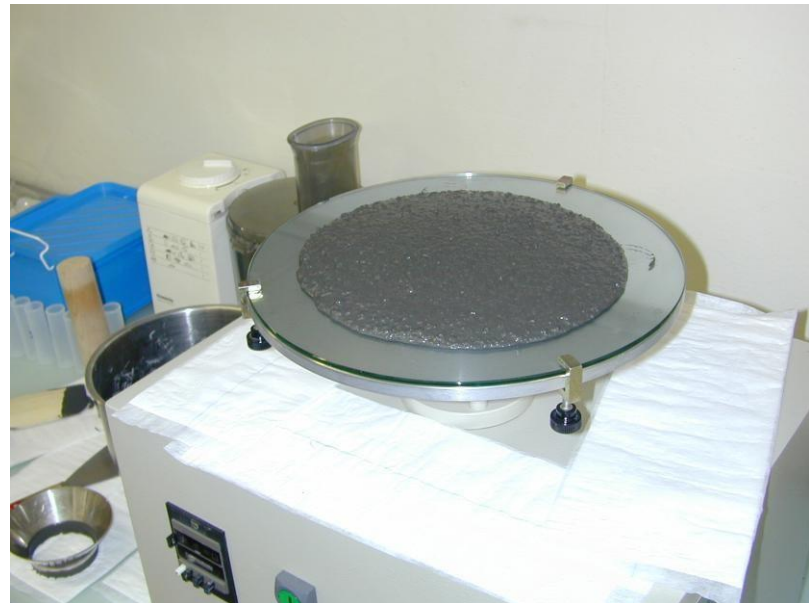
|                     |                     |
|---------------------|---------------------|
| 1.5 cm <            | Graphite Mortar     |
| 1 cm < gap < 1.5 cm | Conventional Mortar |
| < 1 cm              | Fine Mortar         |

## Graphite Milling

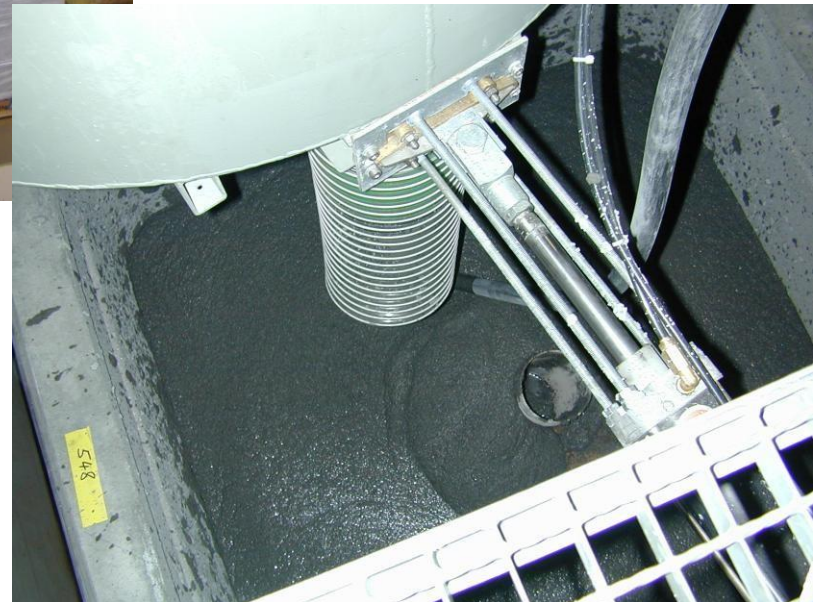
## Dismantling Procedures and Options



Grain size < 5 mm



# Application of Graphite Mortar



- High content of graphite (env. 50%) ✓
- Flowability ✓
- Homogeneity ✓
- No segregation of water ✓
- Moderate temperature development ✓
- Compressive strength above 10MPa ✓
- Low leachability ✓

**Thank you very much  
for your attention!**

